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Modelling of fusion plasma scenarios

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Outline

- the concept of plasma scenario
- the fusion plasma simulator
- main ingredients and approximations
- hierarchy of integrated modelling codes
- examples of ITER scenario simulations

THE CONCEPT OF PLASMA SCENARIO

Scientific objectives

- parameter range exploration
- test of theory/models
- test of techniques
- performance extension

Waveforms

- I_p , n_e , B_t , shape
- heating power
- feedback settings

Machine and plasma parameters

- magnetic equilibrium
- wall condition

Experimental basis

- previous discharges
- scaling laws

Scenario

- set of coherent plasma properties
- machine independent
- reproducible

Actuators

- coils
- H&CD
- torque
- matter injection
- pumping
- impurity seeding

Simulations

- working point (0-D)
- MHD stability domain
- time/profiles evolution
- specific physical phenomena

Adverse events

- MHD / disruptions
- hot spots
- impurity influx
- technical failures

Control & diagnostics

- discharge control
- machine protection
- physics measurements

- **Experimental scenarios exist, but are they extrapolable to ITER/DEMO ?**
 - different **dimensionless** parameter range (ρ^* , v^* , β)
 - different properties of **sources** (e.g., rotation, fast particles)
 - different **control** requirements
 - different level of **self-organization**

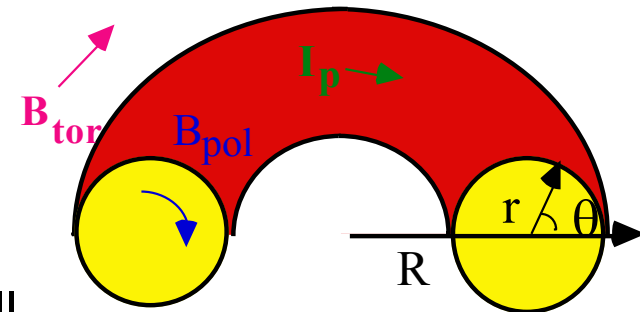
- **Scenario design by integrated tokamak modelling:**
 - a more and more **indispensable tool**, that starts to be efficient
 - neither first-principle nor empirical **transport models** fully reliable
 - **pedestal** is critical: progress on both models and database
 - **edge**: time consuming codes, coupling with core difficult
 - experimental **validation** of code modules
 - interplay with **MHD**: probably the most formidable challenge

- **Challenges related to intrinsic computational complexity:**
 - first-principle **turbulence** codes
 - plasma **edge** codes
 - 3-D non-linear **MHD** codes
 - some **actuator** codes (NBI, ICRH)
 - **Challenges related to code integration:**
 - global **reliability** decreases with number of modules
 - software architecture / use on massively **parallel** computers
 - inclusion of **transients** and **controls**
 - inclusion of all the **transport channels** extremely complex
(electrons, ions, current, particles, momentum, impurities)
- **coordinated EU effort on Integrated Tokamak Modelling**

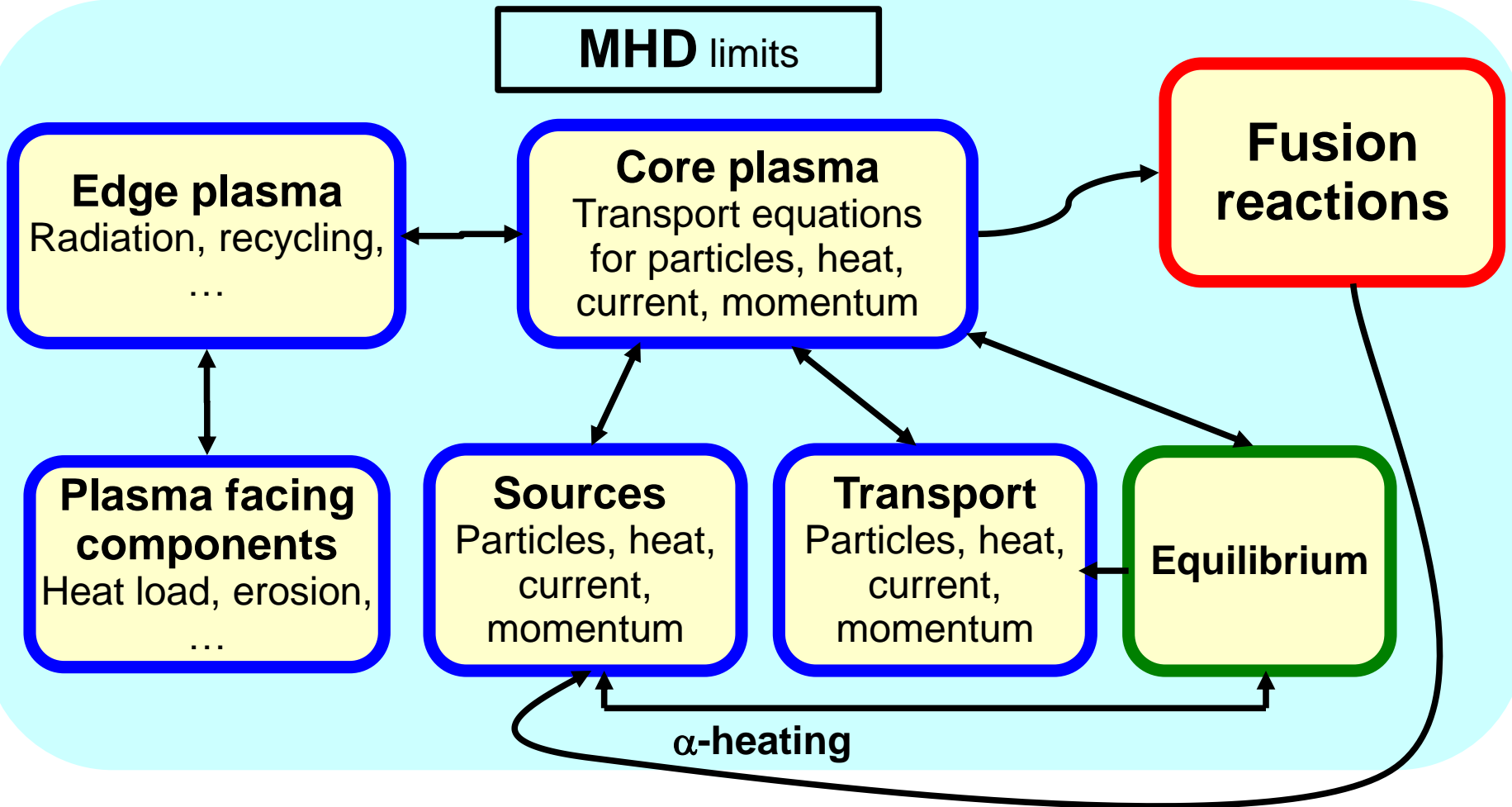
THE FUSION PLASMA SIMULATOR

- Optimization of the operation
- Reactor safety issues
- Design next-step fusion reactors
- Development and validation of physics models
- **extremely difficult and unpractical with a monolithic code (complex Physics/Technology coupling)**
- Generations of modular simulators :
 - **Various levels of approximation available for each element**
 - **Computing resources increase in time**
⇒ **more and more accurate computations**

- **Geometry: magnetic equilibrium**
 - at least 2-D (plasma shaping, separatrix) – 3D for stellarators
 - self-consistent with current and pressure evolution
- **Fluid equations (1-D)**
 - time evolution of n_e , n_i , T_e , T_i , j , V , impurities
- **Sources**
 - heat, injected matter, current, momentum, wall
- **Losses**
 - diffusion/convection of heat and particles
 - pumping / neutralisation
 - radiation (bremsstrahlung, synchrotron, line radiation)
 - viscosity
- **Link to machine data bases** (for application to experiments and validation of the models)



Numerical tokamak: all couplings, all non-linearities



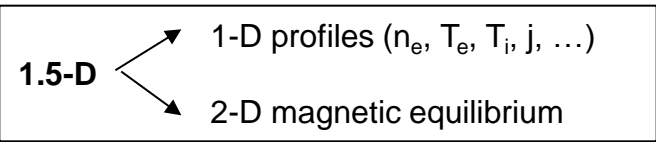
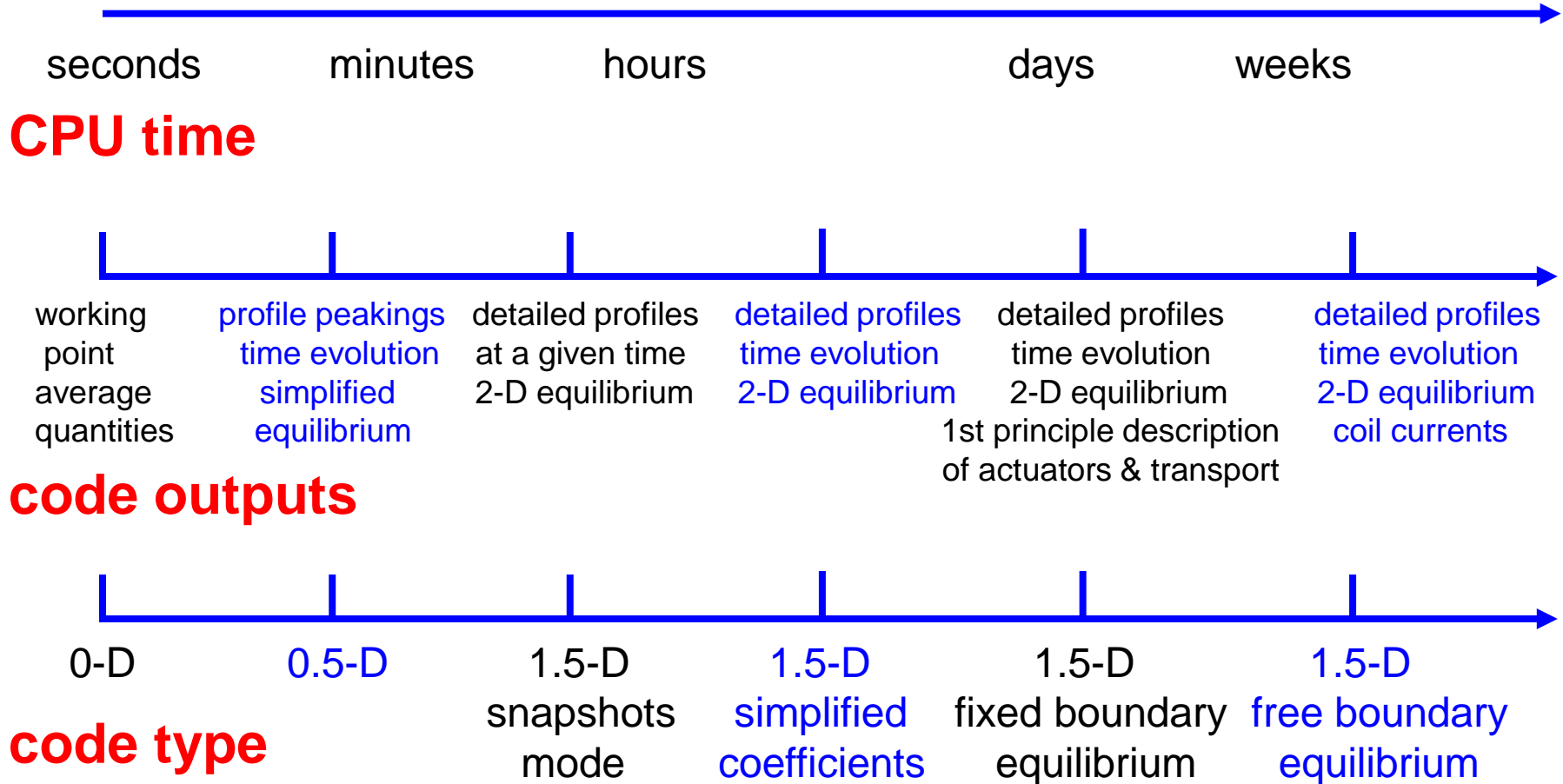
- **Physics**

- wide variety of **physics models** – all the fusion plasma physics
- **integration** of physics and technology (example: antennas)
- to remain as close as possible to the reality of **experiments**
- include description of **actuators, controls, diagnostics**
- compromise between accuracy and **approximations**

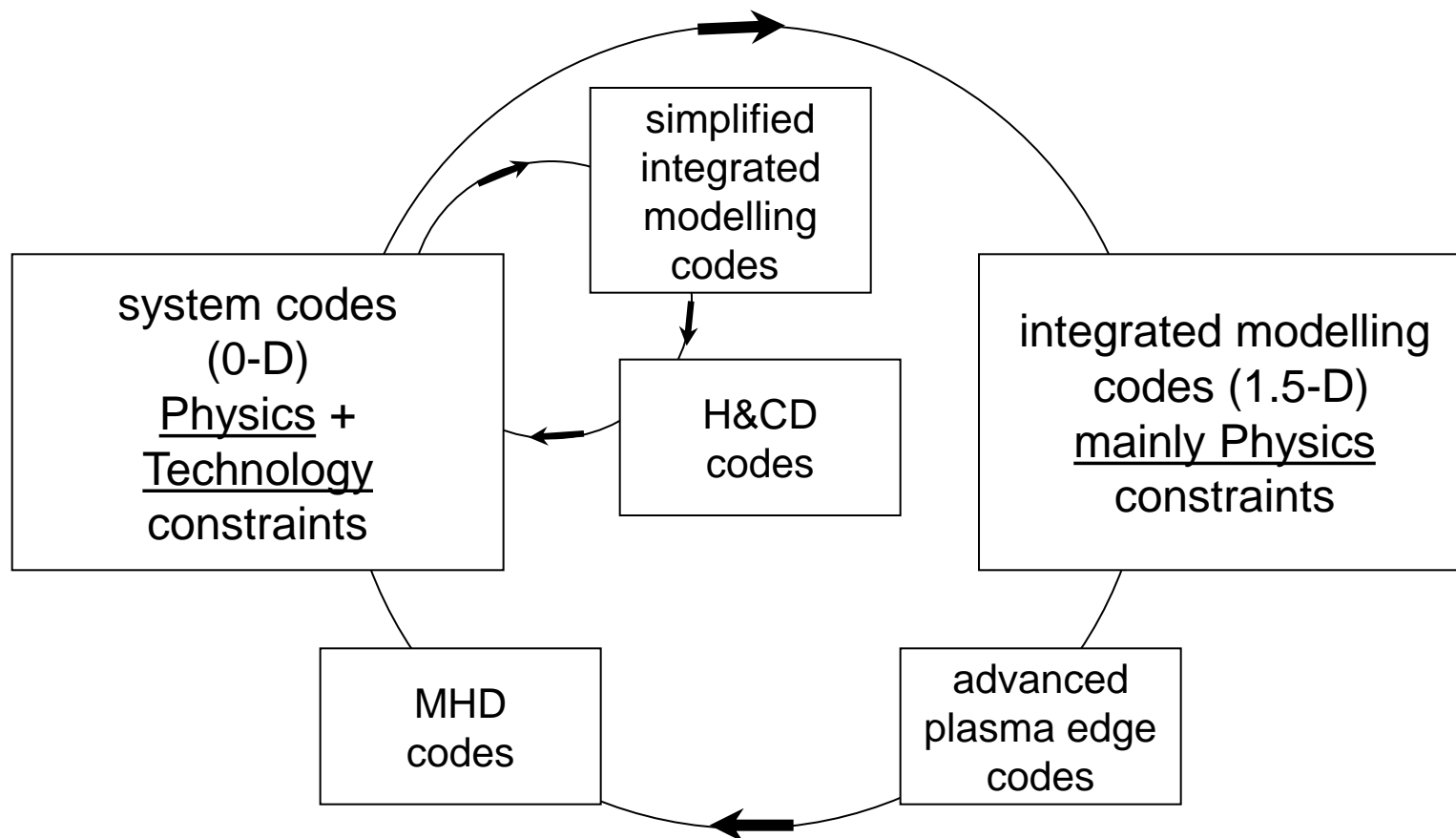
- **Computational**

- integration of **tens of codes (modules)**
- codes of different nature, **language**, generation, speed
- complexity of **software architecture / platform** conception
- speed and memory **optimisation**
- module **reliability**
- **users**: many physicists, with different backgrounds

VARIOUS LEVELS OF SIMULATORS: EXAMPLES



fixed boundary: plasma boundary is given, B field computed in the plasma
free boundary: B field computed in and outside the plasma from coils currents

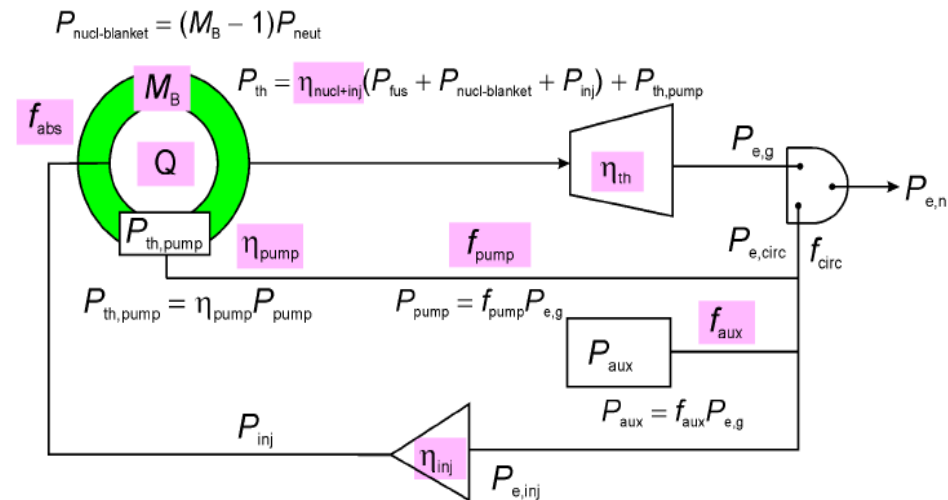


reactor concept optimized by iterations

- computation of a "working point" for space-averaged plasma and machine parameters, with no time evolution
- solution of 0-D **core thermal equilibrium equation**

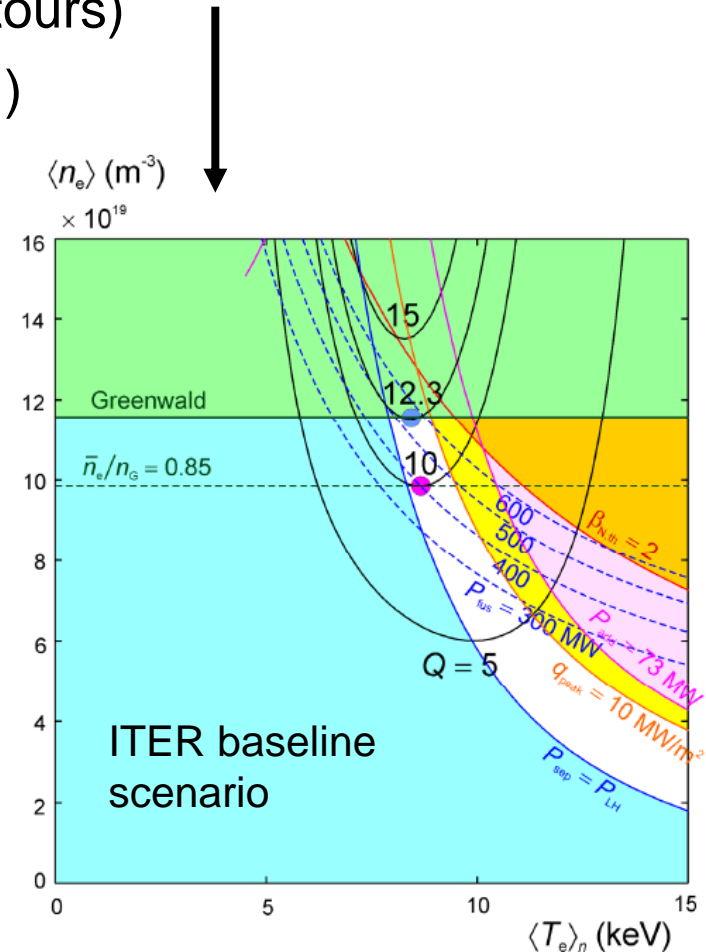
$$\begin{array}{ccccccc}
 P_{\alpha} & + & P_{OH} & + & P_{add} & = & P_{brem} & + & P_{syn} & + & P_{line} & + & P_{cond} \\
 \uparrow & & \uparrow & & \uparrow & & \uparrow & & \uparrow & & \uparrow & & \uparrow \\
 \text{heating} & & \text{alpha} & & \text{ohmic} & & \text{brems-} & & \text{synchrotron} & & \text{atom. line} & & \text{convection} \\
 / \text{losses} & & & & & & \text{strahlung} & & \text{radiation} & & \text{radiation} & & / \text{diffusion} \\
 & & & & \text{add.} & & & & & & & &
 \end{array}$$

- physics constraints on He confinement, heat transport, plasma shape, CD efficiency, density limit, MHD, etc.
- technology constraints on divertor load, blanket properties, pumping, superconducting magnetic field, neutronics, conversion to electric energy, mechanics, etc.



- output: PopCon plots (plasma operation contours)
- example: **HELIOS** code (many other codes...)
(*J. Johner, Fusion Sci. Techn. 59 (2011) 308*)

- possible automatic search of an optimum working point
- input: parameter boundaries and constraints
- example: **PROCESS** code
(*D. Ward, Pl. Phys. Contr. Fus. 52 (2010) 124033*)
- optimisation may include cost !
- these are also called "**system codes**"
- analogous codes exist in USA, Japan, etc.



- **1.5-D codes:** 1-D in space (minor radius coordinate) + 2-D equilibrium
- **output:** full time evolution of equilibria, radial profiles of fluid quantities (density, temperature, current density, plasma momentum) and global quantities (powers, currents, plasma energy, etc.)

Code name	Lab/country	Strong points
ASTRA	IPP / Germany	Many users, flexibility, open structure, reliability
JINTRAC	JET / EU - UK	Validation on JET data, coupling with edge, impurities, pellets
CRONOS	CEA / France	Modularity, H&CD modules, built-in controls, interface
ETS	EU	Integration in a platform of EU codes
TSC / TRANSP	Princeton / USA	Free-boundary capability, H&CD modules, validation on experiments
TOPICS	JAEA / Japan	H&CD modules, validation on Japanese experiments

1.5-D scenario modelling tools. An example: the CRONOS suite of codes

■ Integrated modelling of:

- Heat, particles, rotation ⇒ transport diffusion equations **(1D)**
(heat, matter), source codes
- Current profiles ⇒ current diffusion equation **(1D)**
- Plasma equilibrium ⇒ magnetic equilibrium code **(2D)**

■ Predictive or interpretative modelling:

➤ Interpretative:

- resolution of current diffusion only
- measured electron & ion densities & temperatures

➤ Predictive:

- transport modelling
- initial profiles from model or experiments

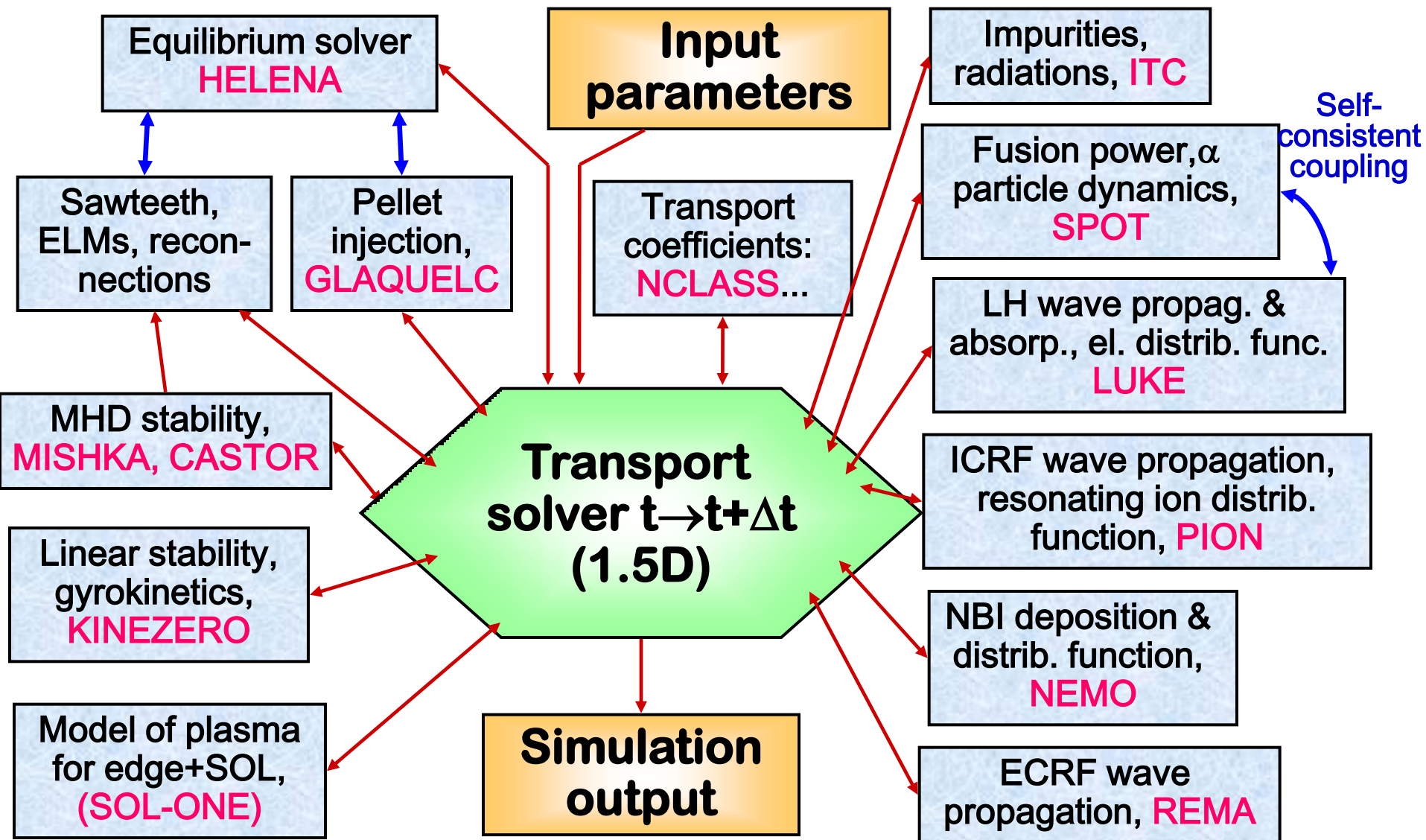


CRONOS

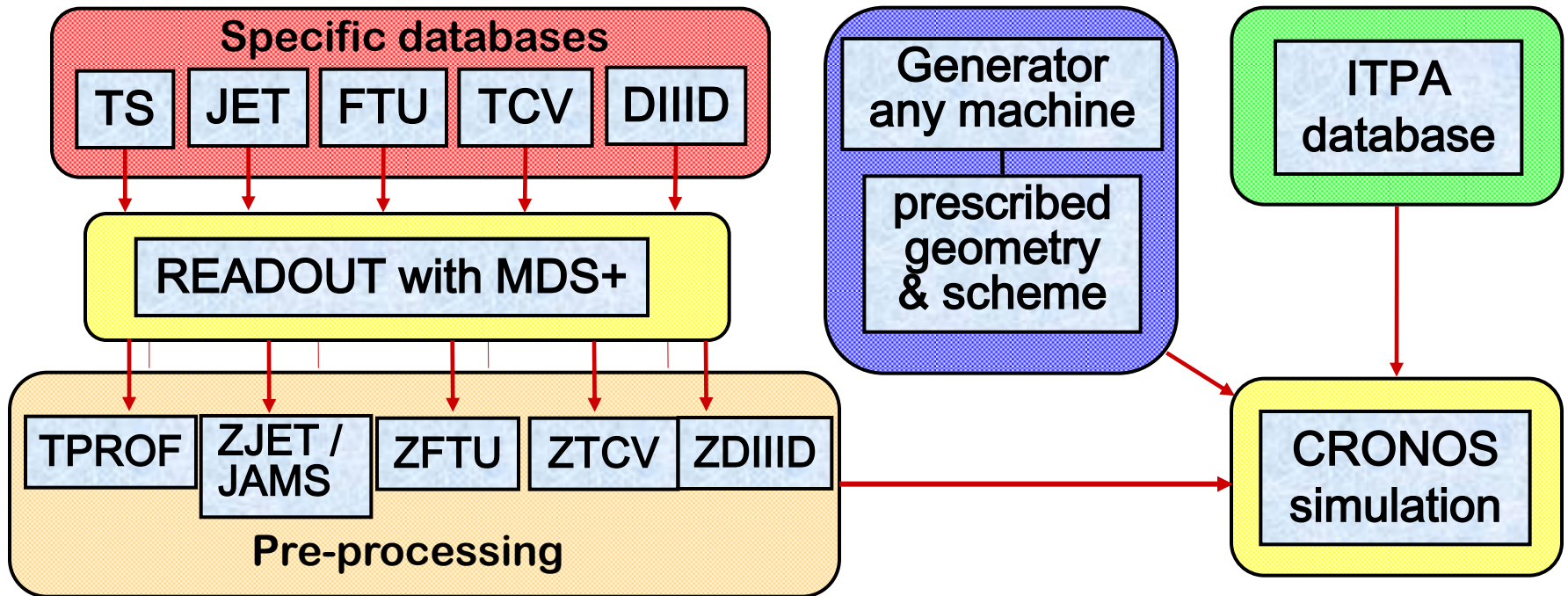
Ref. [J.F. Artaud et al., Nuclear Fusion **50** (2010) 043001]

■ Built-in feedback controls

- ~900 000 lines Fortran 77, ~75 000 lines Fortran 90/95, ~100 000 lines C, ~12 000 lines C++, ~550 000 lines Matlab



User-friendly graphic interface (MATLAB):



➔ Experimental data converted into standard data structure readable by CRONOS.

PION (Ion Cyclotron Heating)

- simplified (1-D) Fokker-Planck code for fast ion tail
- IC wave absorption computed analytically
- Full wave EVE code also available

NEMO/SPOT (Neutral Beam Injection)

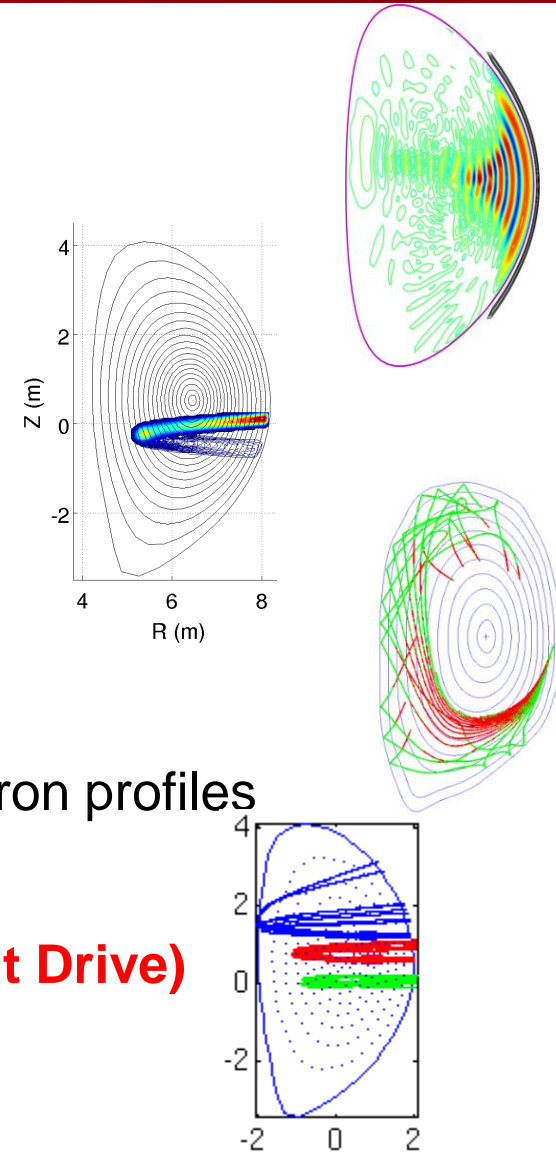
- orbit following MonteCarlo code
- output: heat, particle, current and rotation sources

LUKE (Lower Hybrid Current Drive)

- 3-D ray-tracing coupled to 2D / 3D Fokker-Planck
- output: power deposition, driven current and fast electron profiles

REMA (Electron Cyclotron Heating & Current Drive)

- 3-D ray-tracing with analytical CD efficiency
- output: power deposition and driven current profiles



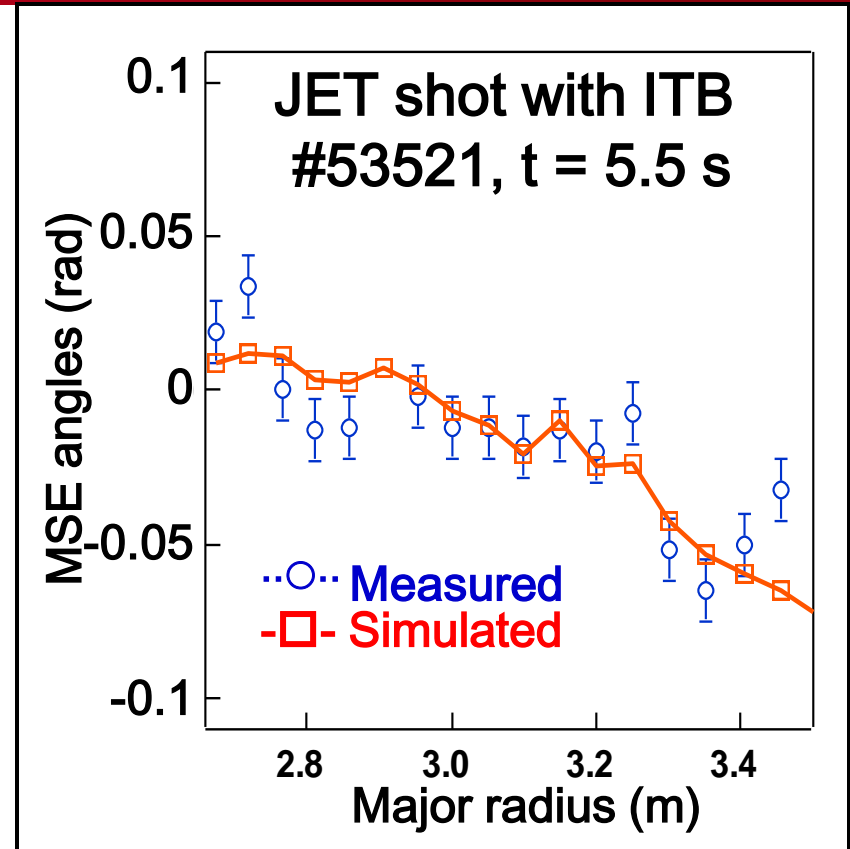
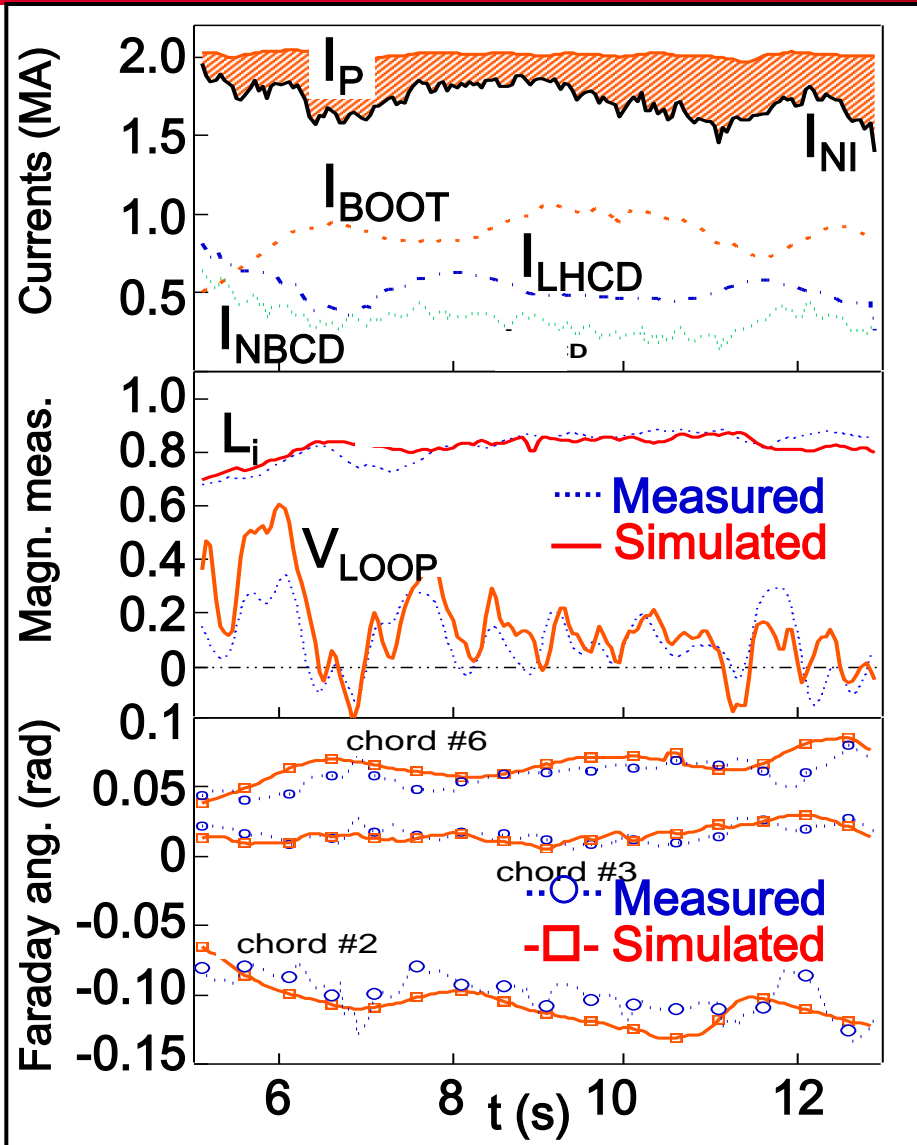
NCLASS (neoclassical transport coefficients)

- multi-species, thermal, axisymmetric plasma, isotropic pressure
- output: resistivity, bootstrap, viscosity, heat diffusivity, etc.

Anomalous transport models (turbulence physics)

- **empirical models:**
 - *kiauto*: reproduces a prescribed 0-D scaling law
 - Bohm/gyro-Bohm (optimised for various machines, with rotation effects)
- **first-principle based models:**
 - GLF23 / TGLF (combines linear gyrokinetic growth rates of ITG/TEM modes with results of the 3D non-linear gyro-fluid code GLF)
 - Weiland (fluid turbulence)
 - Horton (ETG+TEM, critical gradient)
 - CDBM (current diffusive ballooning mode)

CRONOS validation by interpretative simulation of JET experiments

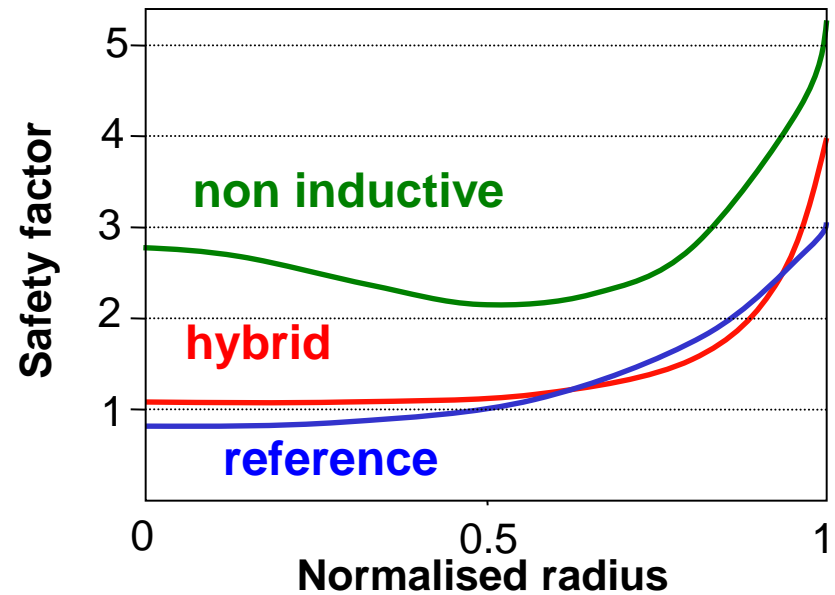
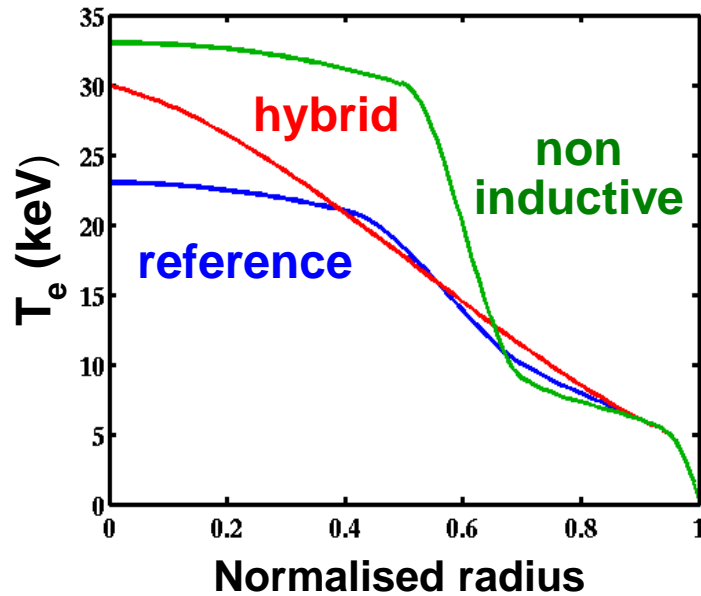


X. Litaudon et al., Nucl. Fusion 44 (2002)

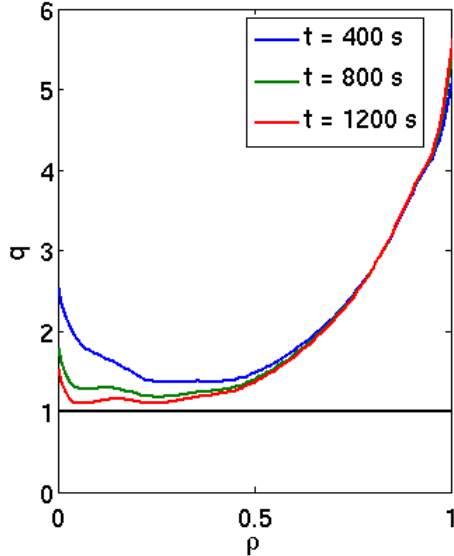
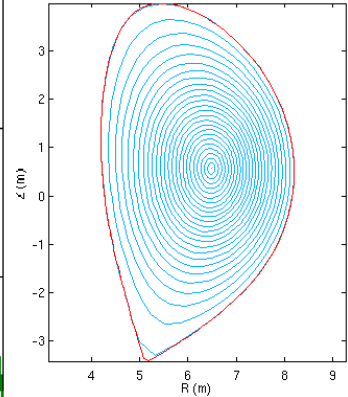
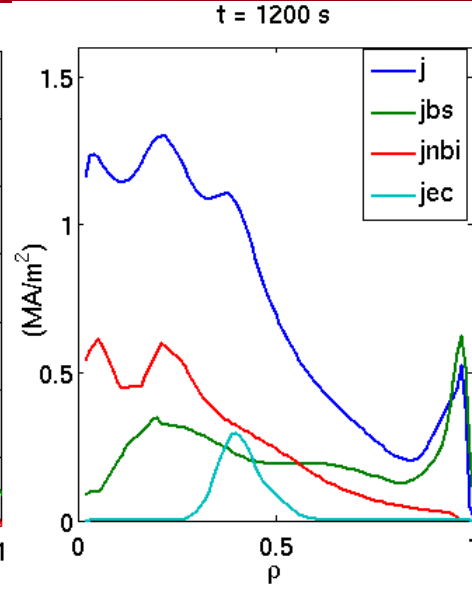
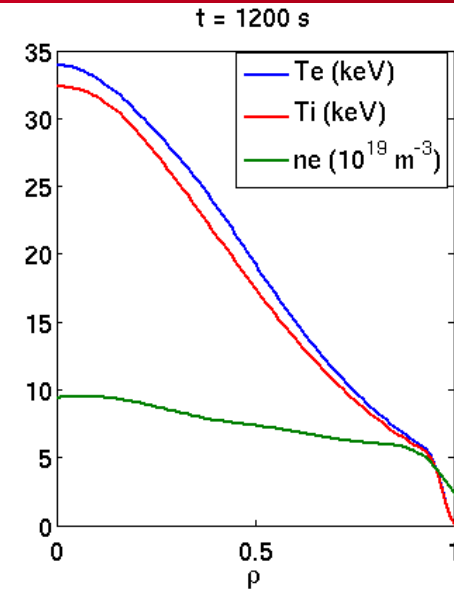
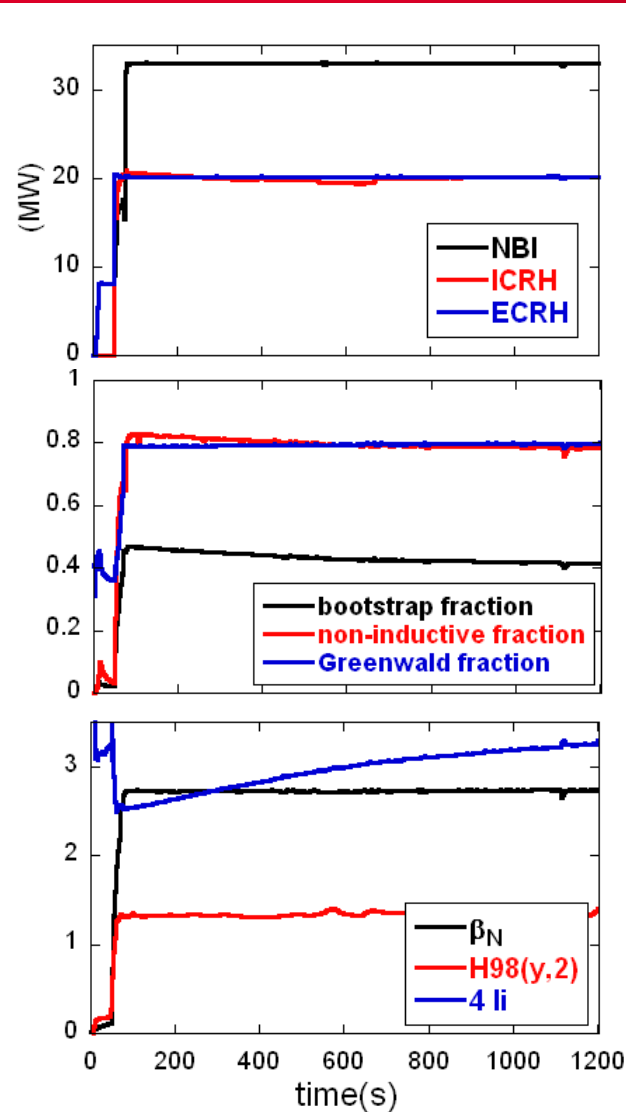
➔ CRONOS is able to reproduce diagnostic signals.

Scenario	I_p (MA)	Duration (s)	Q	Non inductive current
Reference	15	~ 400	10	< 30 %
Hybrid	11 to 13	> 1000	5 to 10	~ 50 %
Non inductive	9	Steady state	~ 5	100 %

$$Q = \frac{P_{fus}}{P_h}$$

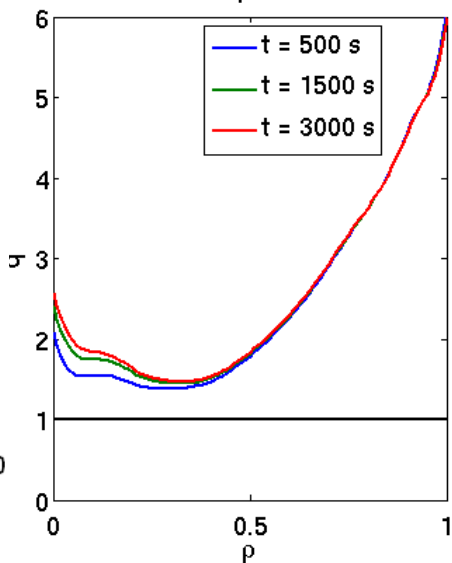
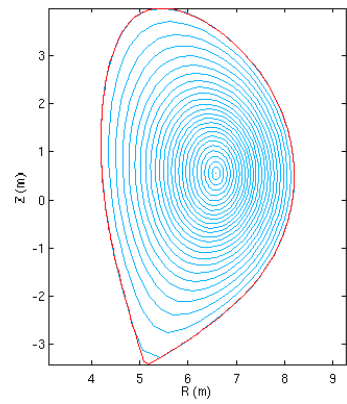
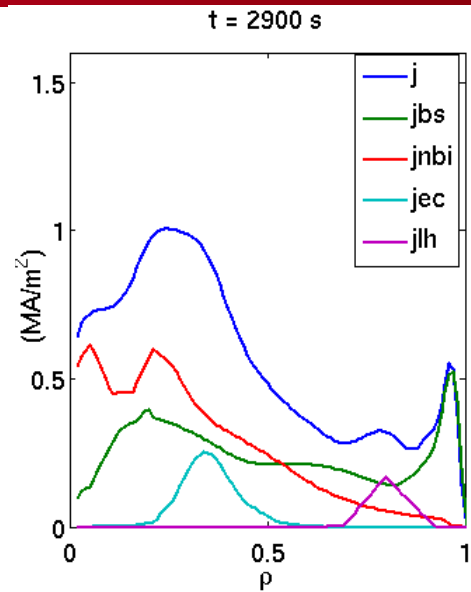
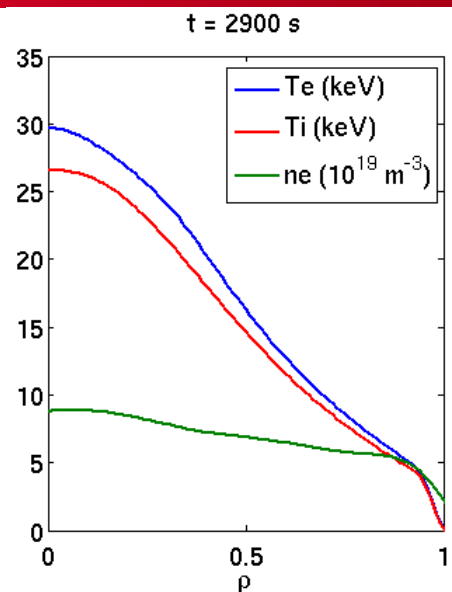
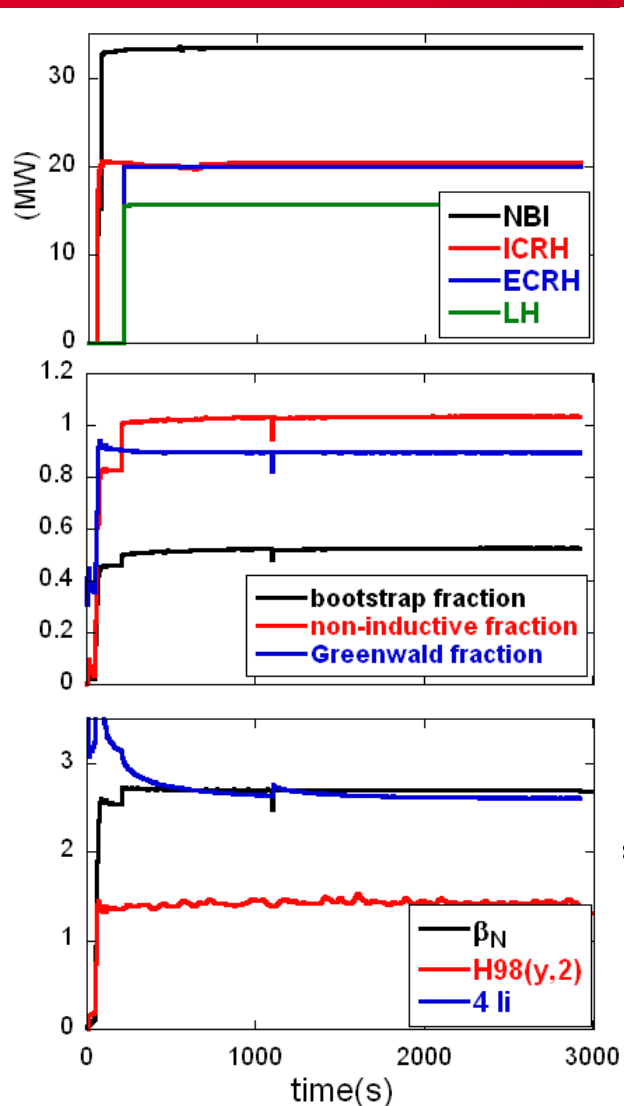


ITER hybrid scenario ($Q \sim 8$ for 1200 s, ideal MHD stable)



Fixed density	$(n_{e0}/\langle n_e \rangle \sim 1.4)$
Fixed T pedestal	$(\sim 5 \text{ keV})$
Fixed H-factor	(~ 1.3)
$I_p = 12 \text{ MA}$	$P_{\text{add}} \sim 73 \text{ MW}$
Bootstrap fraction	$\sim 40\%$
Non-inductive fraction	$\sim 80\%$
Fusion gain Q	~ 8
Fusion Power (MW)	~ 585

ITER steady-state scenario ($Q \sim 5$ for 3000 s, ideal MHD stable)



<u>Fixed density</u>	$(n_{e0}/\langle n_e \rangle \sim 1.4)$
<u>Fixed T pedestal</u>	$(\sim 4 \text{ keV})$
<u>Fixed H-factor</u>	(~ 1.4)
$I_p = 10 \text{ MA}$	$P_{\text{add}} \sim 90 \text{ MW}$
Bootstrap fraction	$\sim 53\%$
Non-inductive fraction	$\sim 100\%$
Fusion gain Q	~ 5
Fusion Power (MW)	~ 425

For steady state

($V_{loop} = 0$):

$$Q = \frac{P_{fus}}{P_h + \frac{\bar{n}_e R I_p}{\langle \gamma_{CD} \rangle} (1 - f_{bs})}$$

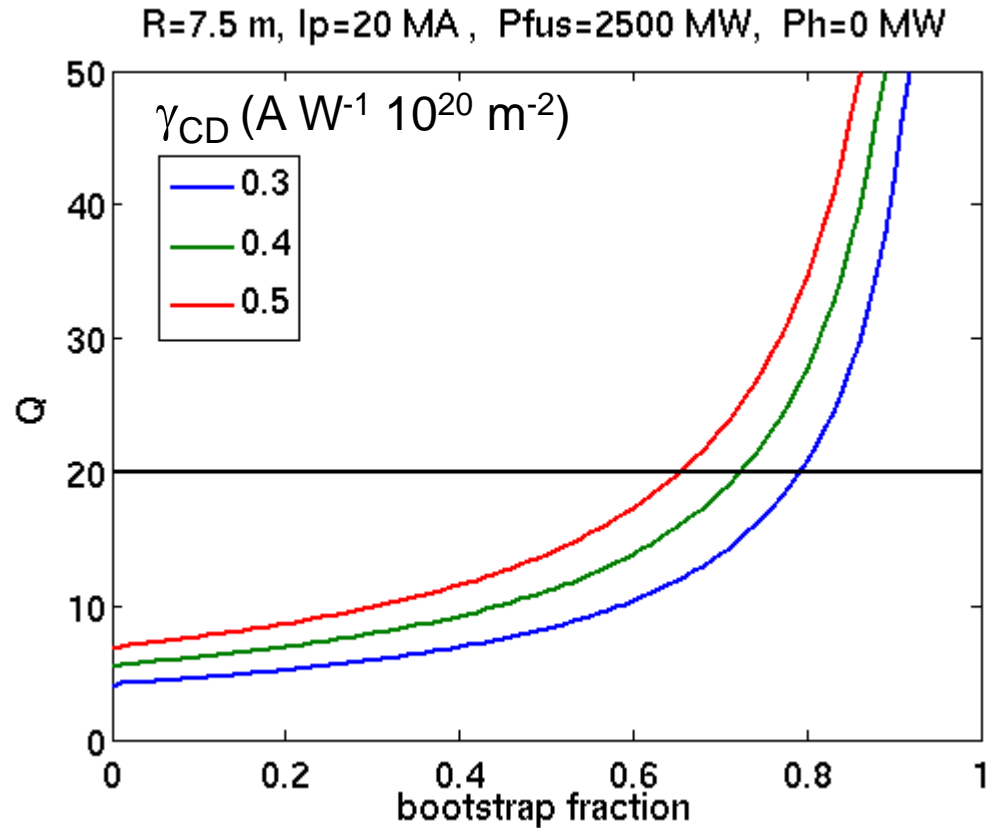
P_h : pure heating power

For realistic CD efficiencies,
large bootstrap fractions
are required

$$Q = \frac{P_{fus}}{P_h + P_{CD}}$$

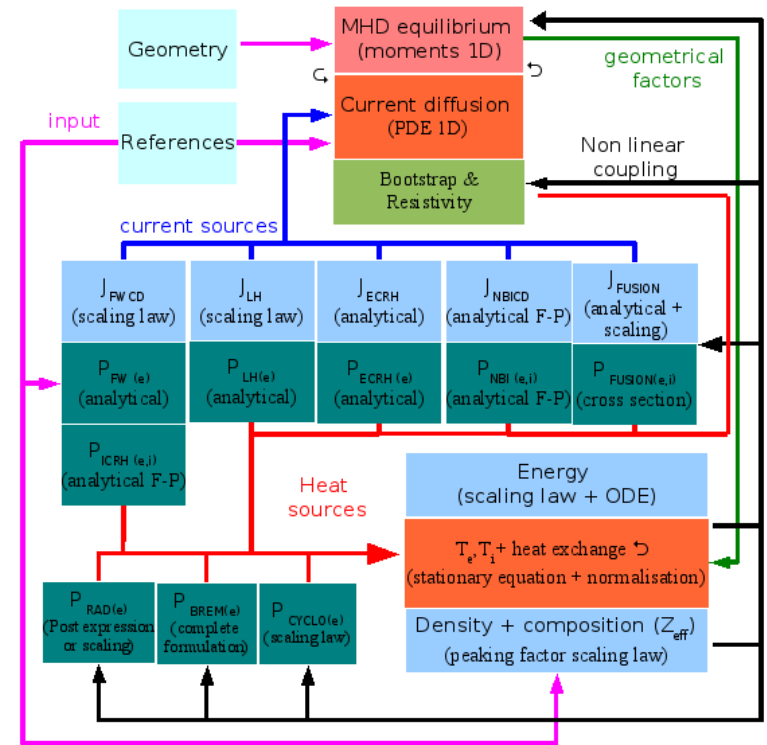
$$\gamma_{CD} = \frac{\bar{n}_e R I_{CD}}{P_{CD}}$$

$$I_p = I_{CD} + I_{bs} \quad (\text{if } V_{loop} = 0)$$



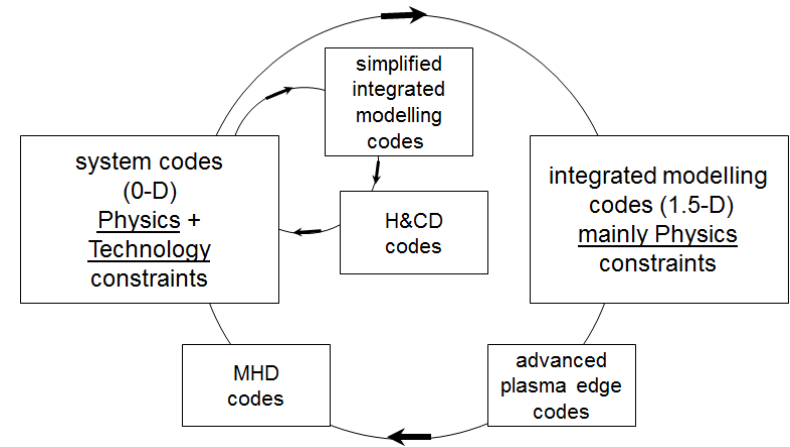
- ✓ METIS is a Plasma Simulator with simplified assumptions
- ✓ METIS is fast : ~ 1 mn per simulation for 300 time slices
- ✓ Mixed 1D and 0D equations
 - Current diffusion 1.5D with equilibrium computed using moment equations
 - Source profiles deduced from simple models
 - Global energy content from 0D ODE (scaling, transients)
 - Temperature profiles : stationary 1D solution, χ scaled to W_{th}
 - All non-linearities solved (dependence of sources on profiles, fusion power, He ash transport, ...)

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- METIS is included in the CRONOS suite of codes (preparation of CRONOS runs)
- Also available as a stand-alone code for Linux, Windows, Mac OSX

- What's a plasma scenario ?
 - a set of coherent plasma properties
 - machine independent
 - reproducible
- Why a tokamak simulator ?
 - Optimization of the operation
 - Reactor safety issues (very first element)
 - Design of next-step fusion machines
 - Development and validation of physics models
- What is needed is a hierarchy of codes/models:
 - **0-D**: fast, give a working point, can be used in optimisation loops
 - **0.5-D**: compute time evolution with simplified profiles and actuators
 - **1.5-D**: full space-time solution, with 2-D equilibria (free or fixed boundary) and detailed description of the actuators (H&CD)



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